

High Density Fuels

OECD-NEA EGATFL 3:rd meeting

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Materials & product performance

Outline

- Requirements
- Metallic Uranium
- Uranium Nitrides
- Uranium Carbides
- Uranium Silicides
- Comparison of HDF:s

Requirements

- Higher uranium density
- Behaviour during normal operation similar or better than UO_2
- Behaviour in water/steam similar or better than UO_2
- High melting point
- If Accident Tolerant Fuel concepts are not required or affected by
 - Law
 - Local codes
 - Cost penalties for non-ATF fuels
- → Any ATF concept will have to benefit the plant owner with a lower **Fuel Cycle Cost** than any alternative

Metallic Uranium

- Pros
 - Uranium loading
 - Thermal conductivity
 - Easy reprocessing
- Cons
 - Reaction with water
 - Melting point
- Conclusion
 - Too volatile in water to be considered as ATF

Uranium Nitrides

- Pros
 - High density, and thereby Uranium loading
 - High thermal conductivity
 - Low Neutron cross section for ^{15}N
 - Known fabrication routes
 - Existing performance data
- Cons
 - Somewhat difficult fabrication
 - Sensitive to water and steam
 - Very High Neutron cross section for ^{14}N
 - ^{15}N enrichment adds complexity and cost
- Other
 - Need/opportunity for ^{15}N recovery during reprocessing

Nitrides –Impact on Fuel Cycle Cost

	UN
Front end	
U mining	Same as UO_2
Conversion	Same as UO_2
U-Enrichment	Same as UO_2
Other isotope enrichment	For the U^{15}N : Difficult and possibly high cost ^{15}N enrichment
Fabrication	Several possible routes, Sensitivity to Oxygen, SPS has demonstrated good pellets. Traditional pressing and sintering possible?
Fabrication Waste	More or less same as UO_2
Operation	Good behaviour likely to enhance economy
Back end	
Reprocessing	Pure UN is fairly easy to convert into $\text{U}_3\text{O}_8 + \text{NH}_3 + \text{H}_2$ by treatment in hot steam. Reprocessing of enriched ^{15}N . When mixed with fission products -TBA
Disposal	TBA

UN manufacturing and reprocessing

- Manufacturing
 - U-metal –hydriding –Nitriding – Spark Plasma Sintering (SPS)
 - Wet sol-gel route to achieve UN +sintering
 - UN:s sensitivity to oxygen and carbon requires protective atmosphere in most process steps
 - Grinding with hydrocarbon cooling (kerosene or white oil)
 - Washing before loading into cladding tubes (e.g. with SCCO_2 , or chlorinated HC:s)
- Reprocessing fairly straight forward
 - Hot steam has proven good results for pure UN
 - Recovery of ^{15}N as enriched ammonia

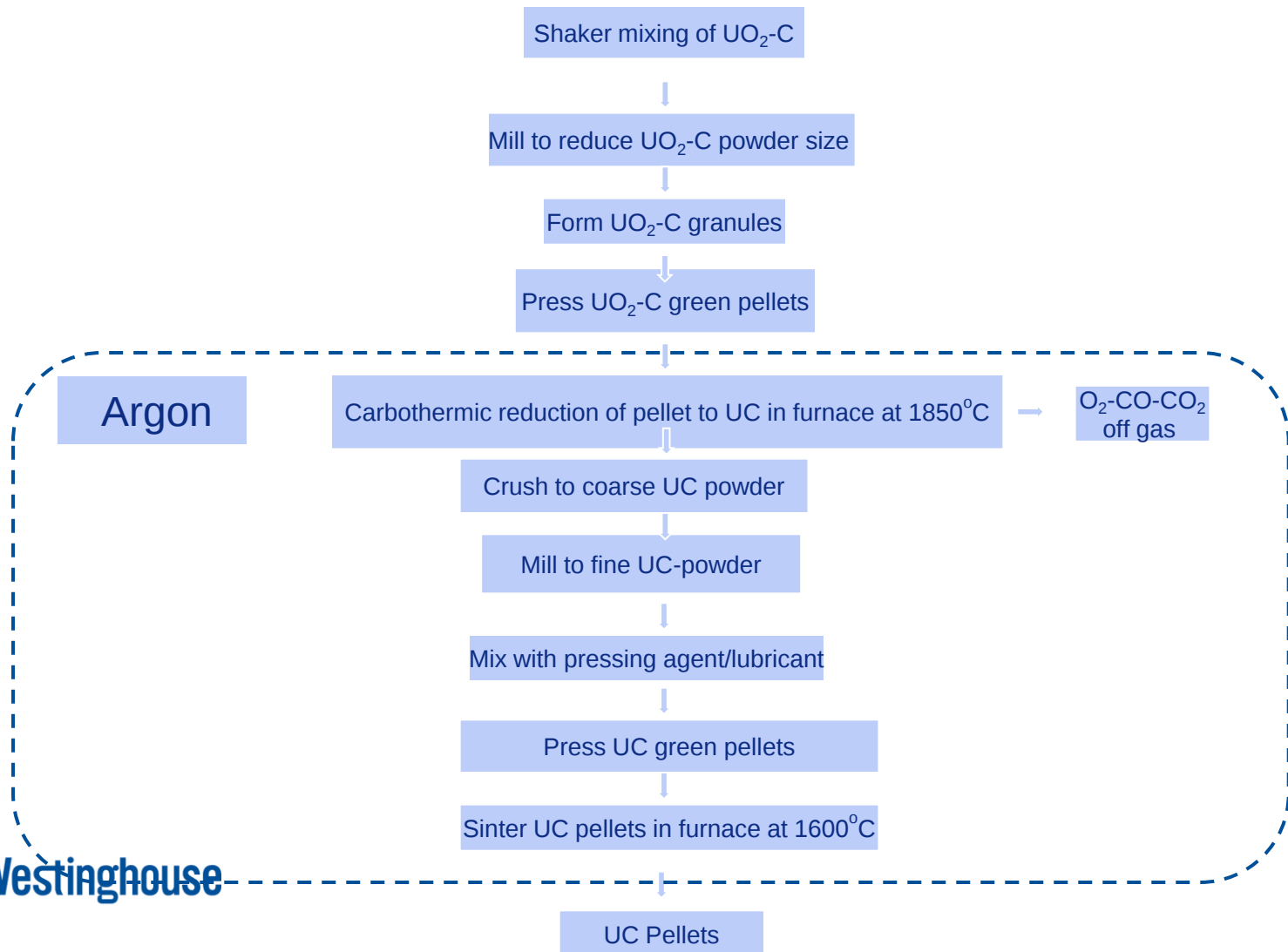
Uranium Carbide

- Pros
 - High Uranium loading
 - Very high thermal conductivity
 - Low release of volatile fission products
- Cons
 - Highly reactive with water and steam, forming mainly methane and hydrogen.
 - Dissociates, forming $UC_{1.1}$ at temperatures above 2250 °C

Carbides –Impact on Fuel Cycle Cost

	UC
Front end	
U mining	Same as UO_2
Conversion	Same as UO_2
U-Enrichment	Same as UO_2
Other isotope enrichment	No
Fabrication	$\text{UO}_2 + \text{C}$ carbothermic reduction. Sensitivity to moisture, Self ignition has been reported. Traditional pressing and sintering possible?
Fabrication Waste	Difficulties due to the volatile nature of the UC
Operation	++ for density, -- for water sensitivity
Back end	
Reprocessing	Pure UC- Direct dissolution in HNO_3 or Oxidizing + dissolution. When mixed with fission products -TBA
Disposal	TBA

UC -Manufacturing



Uranium Silicide

- Pros
 - Good thermal conductivity
 - Relatively stable in water and steam
- Cons
 - Higher thermal neutron cross section than $U^{15}N$, UC and UO_2 , but significantly lower than $U^{14}N$

Silicides –Impact on Fuel Cycle Cost

	U_3Si_2
Front end	
U mining	Same as UO_2
Conversion	Same as UO_2
U-Enrichment	Same as UO_2
Other isotope enrichment	No
Fabrication	Severeral possible routes, direct conversion may be an option
Fabrication Waste	More or less same as UO_2
Operation	+
Back end	
Reprocessing	TBA
Disposal	TBA

U_3Si_2 –Manufacturing and reprocessing

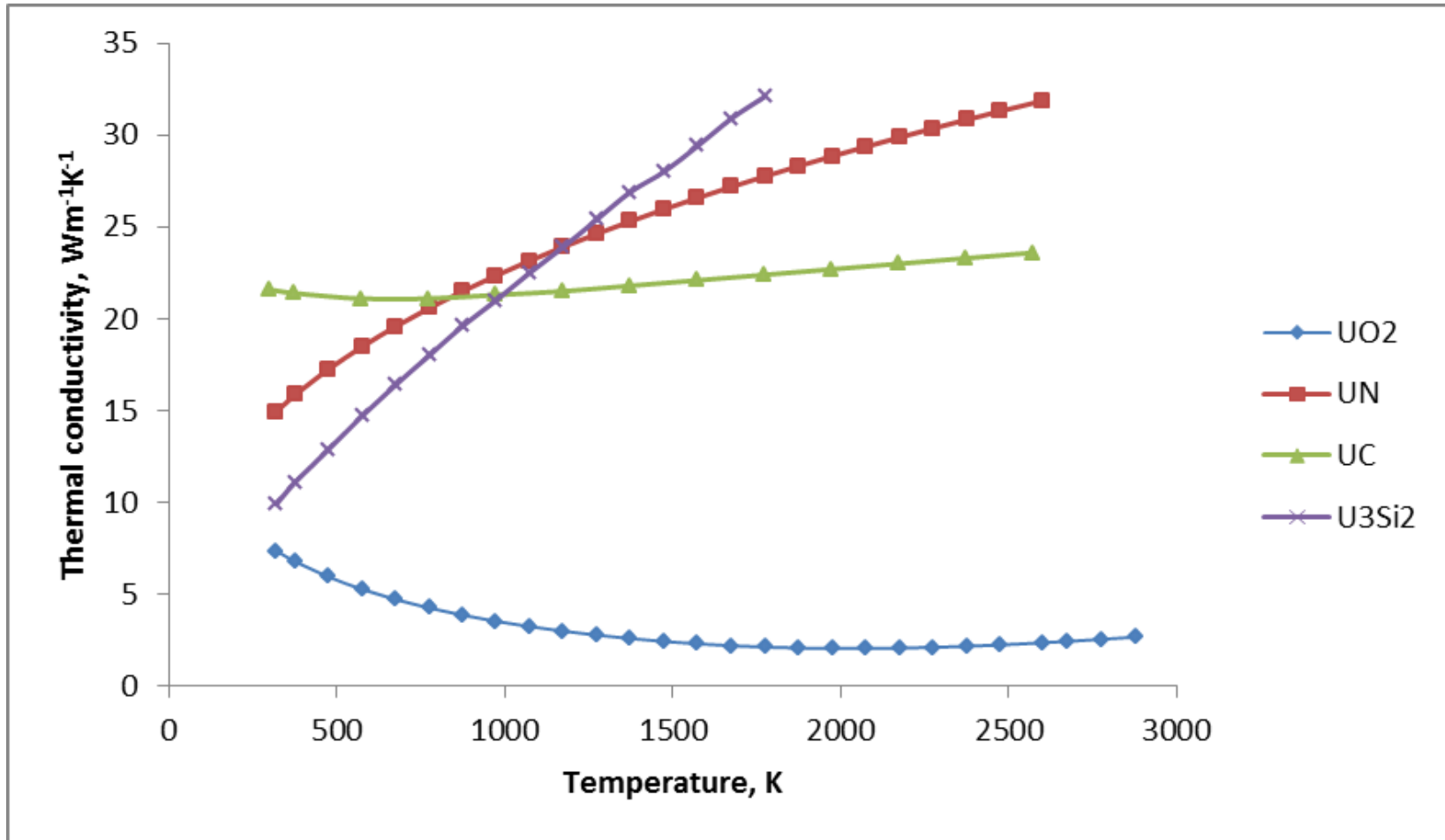
- Manufacturing
 - U-metal –hydride/dehydride mixing with Si, arc-melting-pressing-sintering²⁾
 - Fairly complicated manufacturing process.
 - Grinding in water possible?
- Reprocessing
 - Dissolution in Hg catalyzed HNO_3 , of both Irradiated and unirradiated fuels was demonstrated in the -80:s.

²⁾ Manufacturing route developed at INL, work is ongoing

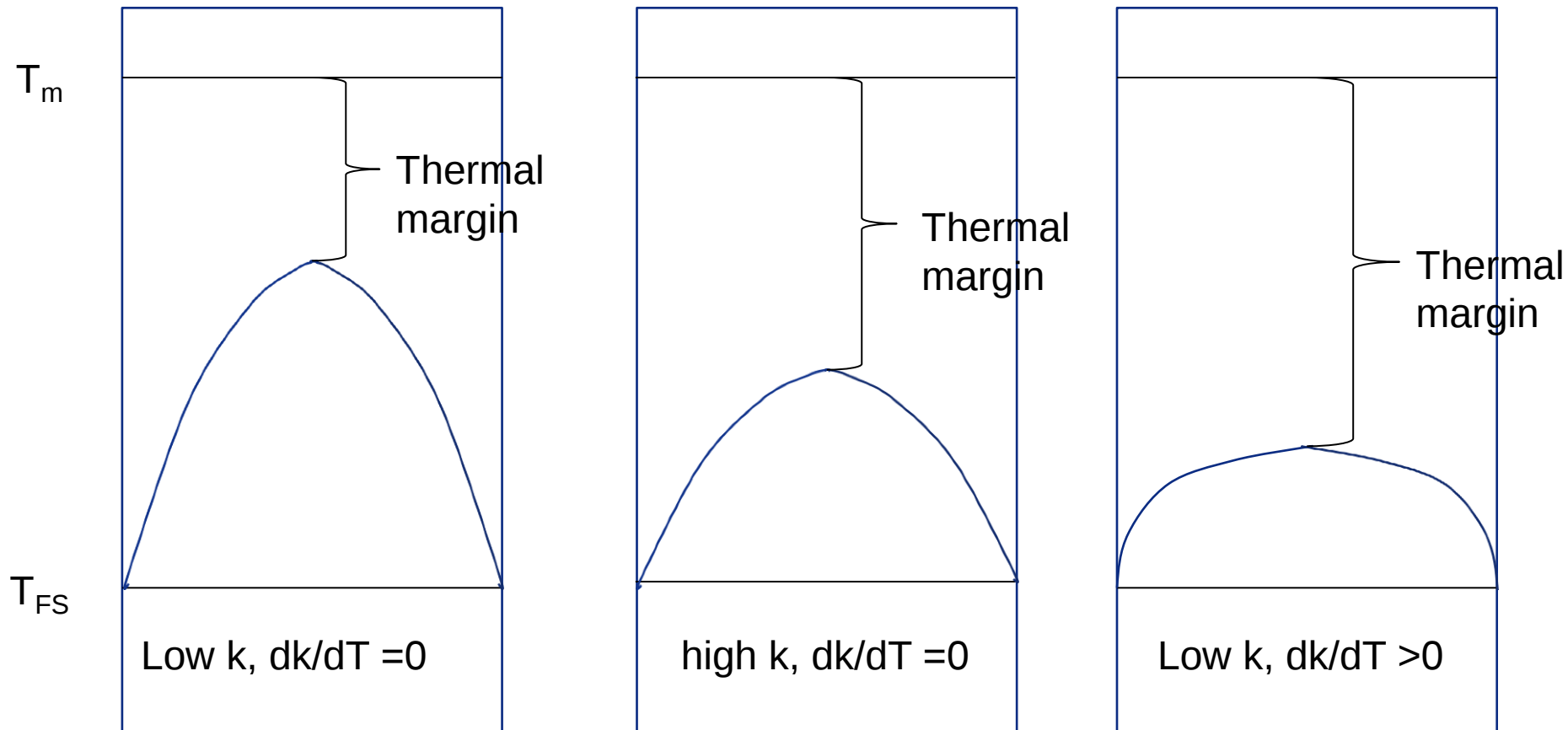
Comparison of different High density Fuel materials

Material	Melting point, °C	Density, g/cm ³	²³⁵ U concentration @ 5% enrichment, g/cm ³	Thermal conductivity, W/mK	Calculated cross section m ⁻¹ x 10 ⁻⁶ Thermal, 2200 m/s
UO ₂	2878	10,96	0,48	See next slide	0,093
U ¹⁵ N	2600	14,31	0,67		0,0082
U ¹⁴ N	2600	14,31	0,68		654
UC	2400	13,6	0,64		1,15
U ₃ Si ₂	1665	12,2	0,56		32,6

Comparison of thermal conductivity



Thermal margin to melt, principle



Conclusions

- Two leading candidates: $U^{15}N$ and U_3Si_2
- Need for common set of material data/calculations for comparison of different fuel concepts